ELSEVIER

Journal of Nuclear Materials 240 (1997) 255–256

journal of nuclear materials

Author index

Abdelouas, A., J.-L. Crovisier, W. Lutze, B. Grambow, J.-C. Dran and R. Müller, Surface layers on a borosilicate nuclear waste glass corroded in $MgCl_2$ solution 240 (1997) 100
Abdou, M.A., see Kumar, A. 240 (1997) 144
Alexander, D.E., Defect production considerations for gamma ray irradiation of reactor pressure vessel steels 240 (1997) 196
Alimov, V.Kh. and B.M.U. Scherzer, Deuterium retention and re-emission from tungsten materials 240 (1996) 75
Allsop, H.A., see Sawicki, J.A. 240 (1996) 22
Ameyama, K., see Nakata, K. 240 (1997) 221

Bagger, C., see Walker, C.T. 240 (1996) 32
Bamberger, C.E., see Shoup, S.S. 240 (1997) 112
Bellanger, G. and J.J. Rameau, Effects of tritiated hydrogen peroxide and chloride on the behavior of N 08932 stainless steel 240 (1996) 1
Bellanger, G. and J.J. Rameau, Interpretation of Inconel 600 behavior in the active region in tritiated water containing chloride–fluoride ions between pH 1 and 3.5 240 (1997) 83
Bharadwaj, S.R., see Mishra, R. 240 (1997) 236
Blank, H., Fast diffusion in dilute binary alloys and its analysis based on Hägg's rule 240 (1997) 169

Catlow, C.R.A., see Nicoll, S. 240 (1997) 185
Chen, K.Q., see Wang, Y.S. 240 (1996) 70
Ciocan, E., see Ignat, M. 240 (1997) 154
Cordfunke, E.H.P., see de Boer, R. 240 (1997) 124
Crovisier, J.-L., see Abdelouas, A. 240 (1997) 100

Dayananda, M.A., see Petri, M.C. 240 (1997) 131
de Boer, R. and E.H.P. Cordfunke, The chemical form of fission product tellurium during reactor accident conditions 240 (1997) 124
Dharwadkar, S.R., see Mishra, R. 240 (1997) 236
Dran, J.-C., see Abdelouas, A. 240 (1997) 100

Forcey, K.S., I. Iordanova and M. Yaneva, The diffusivity and solubility of deuterium in a high chromium martensitic steel 240 (1997) 118
Fukai, K., see Nakata, K. 240 (1997) 221

Gorbatov, V.K., see Grabova, R.B. 240 (1997) 229
Grabova, R.B., V.K. Gorbatov, Z.E. Ostrovsky and R.A. Yuskaev, Microstructural study of an irradiated high-nickel alloy by X-ray line profile analysis and TEM observations 240 (1997) 229
Grambow, B., see Abdelouas, A. 240 (1997) 100
Grimes, R.W., see Nicoll, S. 240 (1997) 185

Haverlock, T.J., see Shoup, S.S. 240 (1997) 112
Hishinuma, A., see Nakata, K. 240 (1997) 221

Ignat, M., E. Ciocan and S. Ion, Method for describing the Zircaloy-4 oxidation using the analytical solution of the oxygen diffusion equation 240 (1997) 154
Ikeda, Y., see Kumar, A. 240 (1997) 144
Ion, S., see Ignat, M. 240 (1997) 154
Iordanova, I., see Forcey, K.S. 240 (1997) 118

Kawamata, K., see Kurumada, A. 240 (1996) 43
Keiser Jr., D.D. and M.C. Petri, Interdiffusion behavior in U–Pu–Zr fuel versus stainless steel couples 240 (1996) 51
Kerkar, A.S., see Mishra, R. 240 (1997) 236
Kim, Y., W.-E. Wang, D.R. Olander and S.K. Yagnik, High-pressure hydriding of Zircaloy 240 (1996) 27
Kumar, A., Y. Ikeda and M.A. Abdou, Experimental validation of radioactivity calculation for candidate fusion materials 240 (1997) 144
Kurome, K., see Watanabe, S. 240 (1997) 251
Kurumada, A., B. McEnaney, T. Oku, K. Kawamata, O. Motojima and N. Noda, Observation and X-ray analysis of the microstructures for a bonding material of C/C composite and copper 240 (1996) 43

Loren Fuller, E. and J.M. Okoh, Kinetics and mechanisms of the reaction of air with nuclear grade graphites: IG-110 240 (1997) 241
Lutze, W., see Abdelouas, A. 240 (1997) 100

Matzke, Hj., see Nicoll, S. 240 (1997) 185
Mayer, M., Codeposition of deuterium with BeO at elevated temperatures 240 (1997) 164
McEnaney, B., see Kurumada, A. 240 (1996) 43
Mishra, R., S.R. Bharadwaj, A.S. Kerkar and S.R. Dharwadkar, Standard Gibbs energy of formation of Cs_2CdI_4 240 (1997) 236
Mogensen, M., see Walker, C.T. 240 (1996) 32
Motojima, O., see Kurumada, A. 240 (1996) 43
Müller, R., see Abdelouas, A. 240 (1997) 100

Nakamura, M., see Watanabe, S. 240 (1997) 251
Nakata, K., K. Fukai, A. Hishinuma and K. Ameyama, Formation and annealing behavior of defect clusters in electron or He-ion irradiated Ti-rich Ti–Al alloys 240 (1997) 221
Nicoll, S., Hj. Matzke, R.W. Grimes and C.R.A. Catlow, The behaviour of single atoms of molybdenum in urania 240 (1997) 185
Noda, N., see Kurumada, A. 240 (1996) 43

Okoh, J.M., see Loren Fuller, E. 240 (1997) 241
Oku, T., see Kurumada, A. 240 (1996) 43
Olander, D.R., see Kim, Y. 240 (1996) 27
Ostrovsky, Z.E., see Grabova, R.B. 240 (1997) 229

Perujo, A., see Serra, E. 240 (1997) 215
Peterson, J.R., see Shoup, S.S. 240 (1997) 112
Petri, M.C. and M.A. Dayananda, Isothermal diffusion in uranium–plutonium–zirconium alloys 240 (1997) 131
Petri, M.C., see Keiser Jr., D.D. 240 (1996) 51

Quan, J.M., see Wang, Y.S. 240 (1996) 70

Rameau, J.J., see Bellanger, G. 240 (1996) 1
Rameau, J.J., see Bellanger, G. 240 (1997) 83
Rest, J., A generalized model for radiation-induced amorphization and crystallization of U_3Si and U_3Si_2 and recrystallization of UO_2 240 (1997) 205

Sakaguchi, N., see Watanabe, S. 240 (1997) 251
Sawicki, J.A. and H.A. Allsop, Evidence of zinc ferrite formation on carbon steel in primary-side coolant with added soluble zinc 240 (1996) 22
Scherzer, B.M.U., see Alimov, V.Kh. 240 (1996) 75
Scholz, R., The effect of cyclic loading on the irradiation hardening of type 316L stainless steel 240 (1997) 161
Serra, E. and A. Perujo, Influence of the surface conditions on permeation in the deuterium–MANET system 240 (1997) 215
Shoji, T., see Yi, Y.S. 240 (1996) 62
Shoup, S.S., C.E. Bamberger, T.J. Haverlock and J.R. Peterson, Aqueous leachability of lanthanide and plutonium titanates 240 (1997) 112
Sun, J.G., see Wang, Y.S. 240 (1996) 70

Takahashi, H., see Watanabe, S. 240 (1997) 251

Walker, C.T., C. Bagger and M. Mogensen, Observations on the release of cesium from UO_2 fuel 240 (1996) 32
Wang, W.-E., see Kim, Y. 240 (1996) 27
Wang, Y.S., K.Q. Chen, C.H. Zhang, J.M. Quan, J.G. Sun and Z.Y. Zhao, The study of bubble formation in 316L stainless steel irradiated with helium ions at 873 K 240 (1996) 70
Watanabe, S., N. Sakaguchi, K. Kurome, M. Nakamura and H. Takahashi, On the mechanism of radiation-induced segregation 240 (1997) 251

Yagnik, S.K., see Kim, Y. 240 (1996) 27
Yaneva, M., see Forcey, K.S. 240 (1997) 118
Yi, Y.S. and T. Shoji, Quantitative evaluation of material degradation of thermally aged duplex stainless steels using chemical immersion test 240 (1996) 62
Yuskaev, R.A., see Grabova, R.B. 240 (1997) 229

Zhang, C.H., see Wang, Y.S. 240 (1996) 70
Zhao, Z.Y., see Wang, Y.S. 240 (1996) 70

ELSEVIER

Journal of Nuclear Materials 240 (1997) 257–261

journal of nuclear materials

Subject index

Actinides *(excludes Plutonium, Thorium and Uranium)*

Surface layers on a borosilicate nuclear waste glass corroded in $MgCl_2$ solution, A. Abdelouas, J.-L. Crovisier, W. Lutze, B. Grambow, J.-C. Dran and R. Müller 240 (1997) 100

Aluminum, Aluminum Alloys and Compounds

Formation and annealing behavior of defect clusters in electron or He-ion irradiated Ti-rich Ti–Al alloys, K. Nakata, K. Fukai, A. Hishinuma and K. Ameyama 240 (1997) 221

Amorphization and Amorphous Materials

Surface layers on a borosilicate nuclear waste glass corroded in $MgCl_2$ solution, A. Abdelouas, J.-L. Crovisier, W. Lutze, B. Grambow, J.-C. Dran and R. Müller 240 (1997) 100

A generalized model for radiation-induced amorphization and crystallization of U_3Si and U_3Si_2 and recrystallization of UO_2, J. Rest 240 (1997) 205

Analytical Instruments and Methods

Evidence of zinc ferrite formation on carbon steel in primary-side coolant with added soluble zinc, J.A. Sawicki and H.A. Allsop 240 (1996) 22

Microstructural study of an irradiated high-nickel alloy by X-ray line profile analysis and TEM observations, R.B. Grabova, V.K. Gorbatov, Z.E. Ostrovsky and R.A. Yuskaev 240 (1997) 229

Beryllium, Beryllium Alloys and Compounds

Codeposition of deuterium with BeO at elevated temperatures, M. Mayer 240 (1997) 164

Carbon

Observation and X-ray analysis of the microstructures for a bonding material of C/C composite and copper, A. Kurumada, B. McEnaney, T. Oku, K. Kawamata, O. Motojima and N. Noda 240 (1996) 43

Kinetics and mechanisms of the reaction of air with nuclear grade graphites: IG-110, E. Loren Fuller and J.M. Okoh 240 (1997) 241

Cavities *(includes Voids, Holes)*

The study of bubble formation in 316L stainless steel irradiated with helium ions at 873 K, Y.S. Wang, K.Q. Chen, C.H. Zhang, J.M. Quan, J.G. Sun and Z.Y. Zhao 240 (1996) 70

Formation and annealing behavior of defect clusters in electron or He-ion irradiated Ti-rich Ti–Al alloys, K. Nakata, K. Fukai, A. Hishinuma and K. Ameyama 240 (1997) 221

Chemical Reactions *(includes Electrochemical and Thermochemical Reactions)*

High-pressure hydriding of Zircaloy, Y. Kim, W.-E. Wang, D.R. Olander and S.K. Yagnik 240 (1996) 27

Interpretation of Inconel 600 behavior in the active region in tritiated water containing chloride–fluoride ions between pH 1 and 3.5, G. Bellanger and J.J. Rameau 240 (1997) 83

Aqueous leachability of lanthanide and plutonium titanates, S.S. Shoup, C.E. Bamberger, T.J. Haverlock and J.R. Peterson 240 (1997) 112

The chemical form of fission product tellurium during reactor accident conditions, R. de Boer and E.H.P. Cordfunke 240 (1997) 124

Method for describing the Zircaloy-4 oxidation using the analytical solution of the oxygen diffusion equation, M. Ignat, E. Ciocan and S. Ion 240 (1997) 154

Kinetics and mechanisms of the reaction of air with nuclear grade graphites: IG-110, E. Loren Fuller and J.M. Okoh 240 (1997) 241

Cladding Materials

High-pressure hydriding of Zircaloy, Y. Kim, W.-E. Wang, D.R. Olander and S.K. Yagnik 240 (1996) 27

Method for describing the Zircaloy-4 oxidation using the analytical solution of the oxygen diffusion equation, M. Ignat, E. Ciocan and S. Ion 240 (1997) 154

Compatibility and Corrosion *(includes Stress Corrosion Cracking)*

Effects of tritiated hydrogen peroxide and chloride on the behavior of N 08932 stainless steel, G. Bellanger and J.J. Rameau 240 (1996) 1

Evidence of zinc ferrite formation on carbon steel in primary-side coolant with added soluble zinc, J.A. Sawicki and H.A. Allsop 240 (1996) 22

Quantitative evaluation of material degradation of thermally aged duplex stainless steels using chemical immersion test, Y.S. Yi and T. Shoji 240 (1996) 62

Surface layers on a borosilicate nuclear waste glass corroded in $MgCl_2$ solution, A. Abdelouas, J.-L. Crovisier, W. Lutze, B. Grambow, J.-C. Dran and R. Müller 240 (1997) 100

Copper, Copper Alloys and Compounds
Observation and X-ray analysis of the microstructures for a bonding material of C/C composite and copper, A. Kurumada, B. McEnaney, T. Oku, K. Kawamata, O. Motojima and N. Noda 240 (1996) 43

Defects and Defect Structures *(excludes by Irradiation)*
The behaviour of single atoms of molybdenum in urania, S. Nicoll, Hj. Matzke, R.W. Grimes and C.R.A. Catlow 240 (1997) 185

Diffusion
Interdiffusion behavior in U–Pu–Zr fuel versus stainless steel couples, D.D. Keiser Jr. and M.C. Petri 240 (1996) 51
The diffusivity and solubility of deuterium in a high chromium martensitic steel, K.S. Forcey, I. Iordanova and M. Yaneva 240 (1997) 118
Isothermal diffusion in uranium–plutonium–zirconium alloys, M.C. Petri and M.A. Dayananda 240 (1997) 131
Method for describing the Zircaloy-4 oxidation using the analytical solution of the oxygen diffusion equation, M. Ignat, E. Ciocan and S. Ion 240 (1997) 154
Fast diffusion in dilute binary alloys and its analysis based on Hägg's rule, H. Blank 240 (1997) 169

Dosimetry
Experimental validation of radioactivity calculation for candidate fusion materials, A. Kumar, Y. Ikeda and M.A. Abdou 240 (1997) 144

Electron Irradiation
Formation and annealing behavior of defect clusters in electron or He-ion irradiated Ti-rich Ti–Al alloys, K. Nakata, K. Fukai, A. Hishinuma and K. Ameyama 240 (1997) 221
On the mechanism of radiation-induced segregation, S. Watanabe, N. Sakaguchi, K. Kurome, M. Nakamura and H. Takahashi 240 (1997) 251

Electron Microscopy
The study of bubble formation in 316L stainless steel irradiated with helium ions at 873 K, Y.S. Wang, K.Q. Chen, C.H. Zhang, J.M. Quan, J.G. Sun and Z.Y. Zhao 240 (1996) 70
Microstructural study of an irradiated high-nickel alloy by X-ray line profile analysis and TEM observations, R.B. Grabova, V.K. Gorbatov, Z.E. Ostrovsky and R.A. Yuskaev 240 (1997) 229

Embrittlement
Quantitative evaluation of material degradation of thermally aged duplex stainless steels using chemical immersion test, Y.S. Yi and T. Shoji 240 (1996) 62
Defect production considerations for gamma ray irradiation of reactor pressure vessel steels, D.E. Alexander 240 (1997) 196

Environmental Effects
Quantitative evaluation of material degradation of thermally aged duplex stainless steels using chemical immersion test, Y.S. Yi and T. Shoji 240 (1996) 62

Fatigue
The effect of cyclic loading on the irradiation hardening of type 316L stainless steel, R. Scholz 240 (1997) 161

Fission Products
Observations on the release of cesium from UO_2 fuel, C.T. Walker, C. Bagger and M. Mogensen 240 (1996) 32
The chemical form of fission product tellurium during reactor accident conditions, R. de Boer and E.H.P. Cordfunke 240 (1997) 124
Standard Gibbs energy of formation of Cs_2CdI_4, R. Mishra, S.R. Bharadwaj, A.S. Kerkar and S.R. Dharwadkar 240 (1997) 236

Fuels and Fuel Elements
Interdiffusion behavior in U–Pu–Zr fuel versus stainless steel couples, D.D. Keiser Jr. and M.C. Petri 240 (1996) 51
A generalized model for radiation-induced amorphization and crystallization of U_3Si and U_3Si_2 and recrystallization of UO_2, J. Rest 240 (1997) 205

Fusion Reactors
Experimental validation of radioactivity calculation for candidate fusion materials, A. Kumar, Y. Ikeda and M.A. Abdou 240 (1997) 144

Gamma Irradiation
Defect production considerations for gamma ray irradiation of reactor pressure vessel steels, D.E. Alexander 240 (1997) 196

Helium
The study of bubble formation in 316L stainless steel irradiated with helium ions at 873 K, Y.S. Wang, K.Q. Chen, C.H. Zhang, J.M. Quan, J.G. Sun and Z.Y. Zhao 240 (1996) 70

Hydrogen and Hydrides *(includes Deuterium and Deuterides)*
Effects of tritiated hydrogen peroxide and chloride on the behavior of N 08932 stainless steel, G. Bellanger and J.J. Rameau 240 (1996) 1
High-pressure hydriding of Zircaloy, Y. Kim, W.-E. Wang, D.R. Olander and S.K. Yagnik 240 (1996) 27

Deuterium retention and re-emission from tungsten materials, V.Kh. Alimov and B.M.U. Scherzer 240 (1996) 75
The diffusivity and solubility of deuterium in a high chromium martensitic steel, K.S. Forcey, I. Iordanova and M. Yaneva 240 (1997) 118
Codeposition of deuterium with BeO at elevated temperatures, M. Mayer 240 (1997) 164
Influence of the surface conditions on permeation in the deuterium–MANET system, E. Serra and A. Perujo 240 (1997) 215

Ion Irradiation
Deuterium retention and re-emission from tungsten materials, V.Kh. Alimov and B.M.U. Scherzer 240 (1996) 75
The effect of cyclic loading on the irradiation hardening of type 316L stainless steel, R. Scholz 240 (1997) 161
Formation and annealing behavior of defect clusters in electron or He-ion irradiated Ti-rich Ti–Al alloys, K. Nakata, K. Fukai, A. Hishinuma and K. Ameyama 240 (1997) 221

Joining
Observation and X-ray analysis of the microstructures for a bonding material of C/C composite and copper, A. Kurumada, B. McEnaney, T. Oku, K. Kawamata, O. Motojima and N. Noda 240 (1996) 43

Mathematical and Computational Methods
Experimental validation of radioactivity calculation for candidate fusion materials, A. Kumar, Y. Ikeda and M.A. Abdou 240 (1997) 144
The behaviour of single atoms of molybdenum in urania, S. Nicoll, Hj. Matzke, R.W. Grimes and C.R.A. Catlow 240 (1997) 185

Microstructure and Texture *(excludes by Irradiation)*
Observation and X-ray analysis of the microstructures for a bonding material of C/C composite and copper, A. Kurumada, B. McEnaney, T. Oku, K. Kawamata, O. Motojima and N. Noda 240 (1996) 43

Moderator and Reflector Materials
Kinetics and mechanisms of the reaction of air with nuclear grade graphites: IG-110, E. Loren Fuller and J.M. Okoh 240 (1997) 241

Molybdenum, Molybdenum Alloys and Compounds
The behaviour of single atoms of molybdenum in urania, S. Nicoll, Hj. Matzke, R.W. Grimes and C.R.A. Catlow 240 (1997) 185

Neutron Irradiation
Experimental validation of radioactivity calculation for candidate fusion materials, A. Kumar, Y. Ikeda and M.A. Abdou 240 (1997) 144
Microstructural study of an irradiated high-nickel alloy by X-ray line profile analysis and TEM observations, R.B. Grabova, V.K. Gorbatov, Z.E. Ostrovsky and R.A. Yuskaev 240 (1997) 229

Nickel, Nickel Alloys and Compounds
Interpretation of Inconel 600 behavior in the active region in tritiated water containing chloride–fluoride ions between pH 1 and 3.5, G. Bellanger and J.J. Rameau 240 (1997) 83
Microstructural study of an irradiated high-nickel alloy by X-ray line profile analysis and TEM observations, R.B. Grabova, V.K. Gorbatov, Z.E. Ostrovsky and R.A. Yuskaev 240 (1997) 229

Nuclear Properties
Experimental validation of radioactivity calculation for candidate fusion materials, A. Kumar, Y. Ikeda and M.A. Abdou 240 (1997) 144

Permeation
Influence of the surface conditions on permeation in the deuterium–MANET system, E. Serra and A. Perujo 240 (1997) 215

Phase Equilibria *(includes Constitution, Phase Stability, Phase Instability)*
Interdiffusion behavior in U–Pu–Zr fuel versus stainless steel couples, D.D. Keiser Jr. and M.C. Petri 240 (1996) 51

Plutonium, Plutonium Alloys and Compounds
Interdiffusion behavior in U–Pu–Zr fuel versus stainless steel couples, D.D. Keiser Jr. and M.C. Petri 240 (1996) 51
Aqueous leachability of lanthanide and plutonium titanates, S.S. Shoup, C.E. Bamberger, T.J. Haverlock and J.R. Peterson 240 (1997) 112
Isothermal diffusion in uranium–plutonium–zirconium alloys, M.C. Petri and M.A. Dayananda 240 (1997) 131

Radiation Effects: Atomic Defects
Defect production considerations for gamma ray irradiation of reactor pressure vessel steels, D.E. Alexander 240 (1997) 196

Radiation Effects: Extended Defects, Microstructures
Microstructural study of an irradiated high-nickel alloy by X-ray line profile analysis and TEM observations, R.B. Grabova, V.K. Gorbatov, Z.E. Ostrovsky and R.A. Yuskaev 240 (1997) 229

Rare Earths
Aqueous leachability of lanthanide and plutonium titanates, S.S. Shoup, C.E. Bamberger, T.J. Haverlock and J.R. Peterson 240 (1997) 112

Redeposition
Codeposition of deuterium with BeO at elevated temperatures, M. Mayer 240 (1997) 164

Safety of Nuclear Reactors
Evidence of zinc ferrite formation on carbon steel in primary-side coolant with added soluble zinc, J.A. Sawicki and H.A. Allsop 240 (1996) 22

Segregation
On the mechanism of radiation-induced segregation, S. Watanabe, N. Sakaguchi, K. Kurome, M. Nakamura and H. Takahashi 240 (1997) 251

Steels, Austenitic
Effects of tritiated hydrogen peroxide and chloride on the behavior of N 08932 stainless steel, G. Bellanger and J.J. Rameau 240 (1996) 1
Quantitative evaluation of material degradation of thermally aged duplex stainless steels using chemical immersion test, Y.S. Yi and T. Shoji 240 (1996) 62
The study of bubble formation in 316L stainless steel irradiated with helium ions at 873 K, Y.S. Wang, K.Q. Chen, C.H. Zhang, J.M. Quan, J.G. Sun and Z.Y. Zhao 240 (1996) 70
The effect of cyclic loading on the irradiation hardening of type 316L stainless steel, R. Scholz 240 (1997) 161
On the mechanism of radiation-induced segregation, S. Watanabe, N. Sakaguchi, K. Kurome, M. Nakamura and H. Takahashi 240 (1997) 251

Steels, Ferritic
Quantitative evaluation of material degradation of thermally aged duplex stainless steels using chemical immersion test, Y.S. Yi and T. Shoji 240 (1996) 62
The diffusivity and solubility of deuterium in a high chromium martensitic steel, K.S. Forcey, I. Iordanova and M. Yaneva 240 (1997) 118
Defect production considerations for gamma ray irradiation of reactor pressure vessel steels, D.E. Alexander 240 (1997) 196
Influence of the surface conditions on permeation in the deuterium–MANET system, E. Serra and A. Perujo 240 (1997) 215

Surface Effects
Deuterium retention and re-emission from tungsten materials, V.Kh. Alimov and B.M.U. Scherzer 240 (1996) 75

Swelling
The study of bubble formation in 316L stainless steel irradiated with helium ions at 873 K, Y.S. Wang, K.Q. Chen, C.H. Zhang, J.M. Quan, J.G. Sun and Z.Y. Zhao 240 (1996) 70

Theory and Modelling
Method for describing the Zircaloy-4 oxidation using the analytical solution of the oxygen diffusion equation, M. Ignat, E. Ciocan and S. Ion 240 (1997) 154
Defect production considerations for gamma ray irradiation of reactor pressure vessel steels, D.E. Alexander 240 (1997) 196
A generalized model for radiation-induced amorphization and crystallization of U_3Si and U_3Si_2 and recrystallization of UO_2, J. Rest 240 (1997) 205
On the mechanism of radiation-induced segregation, S. Watanabe, N. Sakaguchi, K. Kurome, M. Nakamura and H. Takahashi 240 (1997) 251

Thermal Reactors
Evidence of zinc ferrite formation on carbon steel in primary-side coolant with added soluble zinc, J.A. Sawicki and H.A. Allsop 240 (1996) 22
Kinetics and mechanisms of the reaction of air with nuclear grade graphites: IG-110, E. Loren Fuller and J.M. Okoh 240 (1997) 241

Thermal Shock
Observation and X-ray analysis of the microstructures for a bonding material of C/C composite and copper, A. Kurumada, B. McEnaney, T. Oku, K. Kawamata, O. Motojima and N. Noda 240 (1996) 43

Thermodynamic Properties
Standard Gibbs energy of formation of Cs_2CdI_4, R. Mishra, S.R. Bharadwaj, A.S. Kerkar and S.R. Dharwadkar 240 (1997) 236
Kinetics and mechanisms of the reaction of air with nuclear grade graphites: IG-110, E. Loren Fuller and J.M. Okoh 240 (1997) 241

Thermomechanical Treatment
The study of bubble formation in 316L stainless steel irradiated with helium ions at 873 K, Y.S. Wang, K.Q. Chen, C.H. Zhang, J.M. Quan, J.G. Sun and Z.Y. Zhao 240 (1996) 70

Titanium, Titanium Alloys and Compounds
Formation and annealing behavior of defect clusters in electron or He-ion irradiated Ti-rich Ti–Al alloys, K. Nakata, K. Fukai, A. Hishinuma and K. Ameyama 240 (1997) 221

Tritium and Tritides
Effects of tritiated hydrogen peroxide and chloride on the behavior of N 08932 stainless steel, G. Bellanger and J.J. Rameau 240 (1996) 1
Interpretation of Inconel 600 behavior in the active region in tritiated water containing chloride–fluoride ions between pH 1 and 3.5, G. Bellanger and J.J. Rameau 240 (1997) 83
The diffusivity and solubility of deuterium in a high chromium martensitic steel, K.S. Forcey, I. Iordanova and M. Yaneva 240 (1997) 118

Tungsten, Tungsten Alloys and Compounds
Deuterium retention and re-emission from tungsten materials, V.Kh. Alimov and B.M.U. Scherzer 240 (1996) 75

Uranium, Uranium Alloys and Compounds
Observations on the release of cesium from UO_2 fuel, C.T. Walker, C. Bagger and M. Mogensen 240 (1996) 32
Interdiffusion behavior in U–Pu–Zr fuel versus stainless steel couples, D.D. Keiser Jr. and M.C. Petri 240 (1996) 51
Surface layers on a borosilicate nuclear waste glass corroded in $MgCl_2$ solution, A. Abdelouas, J.-L. Crovisier, W. Lutze, B. Grambow, J.-C. Dran and R. Müller 240 (1997) 100
Isothermal diffusion in uranium–plutonium–zirconium alloys, M.C. Petri and M.A. Dayananda 240 (1997) 131
Fast diffusion in dilute binary alloys and its analysis based on Hägg's rule, H. Blank 240 (1997) 169
The behaviour of single atoms of molybdenum in urania, S. Nicoll, Hj. Matzke, R.W. Grimes and C.R.A. Catlow 240 (1997) 185
A generalized model for radiation-induced amorphization and crystallization of U_3Si and U_3Si_2 and recrystallization of UO_2, J. Rest 240 (1997) 205

Wastes
Surface layers on a borosilicate nuclear waste glass corroded in $MgCl_2$ solution, A. Abdelouas, J.-L. Crovisier, W. Lutze, B. Grambow, J.-C. Dran and R. Müller 240 (1997) 100
Aqueous leachability of lanthanide and plutonium titanates, S.S. Shoup, C.E. Bamberger, T.J. Haverlock and J.R. Peterson 240 (1997) 112

Zirconium, Zirconium Alloys and Compounds
High-pressure hydriding of Zircaloy, Y. Kim, W.-E. Wang, D.R. Olander and S.K. Yagnik 240 (1996) 27
The chemical form of fission product tellurium during reactor accident conditions, R. de Boer and E.H.P. Cordfunke 240 (1997) 124
Isothermal diffusion in uranium–plutonium–zirconium alloys, M.C. Petri and M.A. Dayananda 240 (1997) 131
Method for describing the Zircaloy-4 oxidation using the analytical solution of the oxygen diffusion equation, M. Ignat, E. Ciocan and S. Ion 240 (1997) 154
Fast diffusion in dilute binary alloys and its analysis based on Hägg's rule, H. Blank 240 (1997) 169